

Carolina Power & Light Company Harris Nuclear Plant PO Box 165 New Hill NC 27562

AUG 1 8 1997

U.S. Nuclear Regulatory Commission ATTN: NRC Document Control Desk Washington, DC 20555 Serial: HNP-97-159 10CFR50.73

SHEARON HARRIS NUCLEAR POWER PLANT UNIT 1 DOCKET NO. 50-400 LICENSE NO. NPF-63

LICENSEE EVENT REPORT 97-019-00

Sir or Madam:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report describes a Turbine Trip/Reactor Trip caused by a loss of generator excitation, followed by an automatic actuation of the Auxiliary Feedwater System.

Sincerely,

J. W. Donahue

Director of Site Operations

Harris Plant

MV

C:

Enclosure

Mr. J. B. Brady (HNP Senior NRC Resident)

Mr. L. A. Reyes (NRC Regional Administrator, Region II)

Mr. V. L. Rooney (NRC - NRR Project Manager)

9708250133 970818 PDR ADDCK 05000400 S PDR



TEXA

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9708250133 DOC.DATE: 97/08/18 NOTARIZED: NO DOCKET # FACIL:50-400 Shearon Harris Nuclear Power Plant, Unit 1, Carolina 05000400

AUTH. NAME . AUTHOR AFFILIATION

VERRILLI,M. Carolina Power & Light Co. DONAHUE, J.W. Carolina Power & Light Co.

RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 97-019-00:on 970720, turbine trip/reactor trip occurred. Caused by three phase fault collapsed excitation field in main generator, resulting in generator lockout. Exciter rotor assembly replaced. W/970818 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR | ENCL | SIZE: TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: Application for permit renewal filed.

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	NRR/DE/ECGB	1	1	NRR/DE/EELB	1	1	
	NRR/DE/EMEB	1	1	NRR/DRCH/HHFB	1	1	
	NRR/DRCH/HICB	1	1	NRR/DRCH/HOLB	1	1	
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	NRR/DSSA/SPLB	1	1	NRR/DSSA/SRXB	1	1	
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EXTERNAL:	L ST LOBBY WARD	1	1	LITCO BRYCE, J P	1	1	
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NRC FORM 366

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104 **EXPIRES 04/30/98**

ESTIMATEO BUROEN PER RESPONSE TO COMPLY WITH THIS MANOATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTEO LESSONS LEARNEO ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND

RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION,

WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-

FACILITY NAME (1)

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, OC 20503. DOCKET NUMBER (2)

Harris Nuclear Plant Unit-1

50-400

1 OF 2

TITLE (4)

Turbine Trip/Reactor Trip due to failure of generator exciter.

YEAR 97	YEAR 97 -	SEQUENTIAL NUMBER - 019	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY		DOCKET NUMBER
97	97 -	- 019	00				1		
			00	8	18	97	FACILITY	NAME	O5000
	THIS RE	PORT IS SUBM	ITTED PU	RSUANT	TO THE	REQUI	REMENT	S OF 10 CFR 5: (Check	cone or more) (11)
1	20.2201(b)			20.2203(a)(2)(v)				50.73(a)(2)(i)	50.73(a)(2)(viii)
	20.2	203(a)(1)	20.2203(a)(3)(i)				50.73(a)(2)(ii)	50.73(a)(2)(x)	
100%	20.2203(a)(2)(i)			20.2203(a)(3)(ii)				50.73(a)(2)(iii)	73.71
	20.2	203(a)(2)(ii)		20.2203	3(a)(4)		X	50.73(a)(2)(iv)	OTHER
					50.36(c)(1)			50.73(a)(2)(v)	Specify in Abstract below
	20.2203(a)(2)(iv)			50.36(c)(2)				50.73(a)(2)(vii)	or in NRC Form 366A
	1 100%	1 20.2 20.2 100% 20.2 20.2 20.2	1 20.2201(b) 20.2203(a)(1) 20.2203(a)(2)(i) 20.2203(a)(2)(ii) 20.2203(a)(2)(iii)	1 20.2201(b) 20.2203(a)(1) 20.2203(a)(2)(ii) 20.2203(a)(2)(iii) 20.2203(a)(2)(iii) 20.2203(a)(2)(iiv)	1 20.2201(b) 20.2203 20.2203(a)(1) 20.2203 20.2203(a)(2)(i) 20.2203 20.2203(a)(2)(ii) 20.2203 20.2203(a)(2)(iii) 50.36(c 20.2203(a)(2)(iv) 50.36(c	1 20.2201(b) 20.2203(a)(2)(v) 20.2203(a)(1) 20.2203(a)(3)(i) 20.2203(a)(2)(i) 20.2203(a)(3)(i) 20.2203(a)(2)(ii) 20.2203(a)(4) 20.2203(a)(2)(iii) 50.36(c)(1) 20.2203(a)(2)(iv) 50.36(c)(2)	1 20.2201(b) 20.2203(a)(2)(v) 20.2203(a)(1) 20.2203(a)(3)(i) 20.2203(a)(2)(i) 20.2203(a)(3)(ii) 20.2203(a)(2)(ii) 20.2203(a)(4) 20.2203(a)(2)(iii) 50.36(c)(1) 20.2203(a)(2)(iv) 50.36(c)(2)	1 20.2201(b) 20.2203(a)(2)(v) 100% 20.2203(a)(2)(i) 20.2203(a)(3)(ii) 20.2203(a)(2)(ii) 20.2203(a)(3)(iii) 20.2203(a)(2)(iii) 20.2203(a)(4) X 20.2203(a)(2)(iii) 50.36(c)(1) 20.2203(a)(2)(iv) 50.36(c)(2)	20.2203(a)(1) 20.2203(a)(3)(i) 50.73(a)(2)(ii) 20.2203(a)(2)(ii) 50.73(a)(2)(iii) 20.2203(a)(2)(iii) 20.2203(a)(4) X 50.73(a)(2)(iv) 20.2203(a)(2)(iii) 50.36(c)(1) 50.73(a)(2)(v)

LICENSEE CONTACT FOR THIS LER (12)

NAME

TELEPHONE NUMBER (Include Area Code)

Michael Verrilli Sr. Analyst - Licensing

(919) 362-2303

AUSE SYSTEM COMPONENT MANUFACTURER		REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACT	REPORTABLI TO NPRDS				
E TL	EXC	W123	Υ									
		SUPPLEMENTA	AL REPORT EXPE	CTED (14)			EVE	PECTED	MONTH	DAY		YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

At approximately 0256 hours on July 20, 1997, with the plant at approximately 100% power in mode 1, a Turbine Trip/Reactor Trip occurred due to a generator lock-out resulting from loss of the main generator excitation field. As a result of the trip, the Auxiliary Feedwater (AFW) System automatically started as required to provide feedwater flow to the Steam Generators (S/G). Operators in the main control room immediately implemented Emergency Operating Procedure (EOP Path-1) and upon verification that a safety injection was not required and had not occurred, entered the Reactor Trip Response procedure (EOP-EPP-004) to stabilize the plant. Approximately 5 minutes after the plant trip, AFW flow to the S/Gs was secured in order to return Reactor Coolant System (RCS) temperature back to 557 degrees (RCS temperature reached approximately 548 degrees during AFW operation). Various plant components not required for plant recovery were observed tripped in the main control room due to the electrical transient involved in the plant trip. Automatic protection and safeguards systems functioned as required and the plant was stabilized in mode 3 (Hot Standby).

The cause of the Turbine Trip/Reactor trip was a three phase fault that collapsed the excitation field in the main generator, resulting in a generator lockout. Investigation could not conclusively determine the reason for the electrical fault in the generator exciter. However, the most probable cause appears to be entry of foreign material into the exciter housing that caused multiple shorts on the exciter diode wheel.

Corrective actions for this event included replacement of the exciter rotor assembly and inspections to ensure no foreign material was present in the exciter and exciter housing area. Additional actions will include strengthening the plant's Foreign Material Exclusion Program with regards to the Generator Exciter. This will be performed via a revision to Maintenance Procedure, PM-E0045 "Generator Exciter Cleaning, Inspection, and Testing" to specify cleanliness close-out requirements for the Generator Exciter.

FACILITY NAME (1)	DOCKET		LER NUMBER ((6)		PAGE (3)	
	50.400	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		0.5	
Shearon Harris Nuclear Plant - Unit #1	50-400	97	019	00	2	OF	2

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT DESCRIPTION:

At approximately 0256 hours on July 20, 1997, with the plant at approximately 100% power in mode 1, a Turbine Trip/Reactor Trip occurred due to a generator lock-out resulting from loss of the main generator excitation field. As a result of the trip, the Auxiliary Feedwater (AFW) System automatically started as required to provide feedwater flow to the Steam Generators (S/G). Operators in the main control room immediately implemented Emergency Operating Procedure (EOP Path-1) and upon verification that a safety injection was not required and had not occurred, entered the Reactor Trip Response procedure (EOP-EPP-004) to stabilize the plant. Approximately 5 minutes after the plant trip, AFW flow to the S/Gs was secured in order to return Reactor Coolant System (RCS) temperature back to 557 degrees (RCS temperature reached approximately 548 degrees during AFW operation). Various plant components were observed tripped in the main control room due to the electrical transient involved in the plant trip. Automatic protection and safeguards systems functioned as required and the plant was stabilized in mode 3 (Hot Standby).

CAUSE:

The cause of the Turbine Trip/Reactor trip was a three phase fault that collapsed the excitation field in the main generator, resulting in a generator lockout. Investigation could not conclusively determine the reason for the electrical fault in the generator exciter. However, the most probable cause appears to be entry of foreign material into the exciter housing that caused multiple shorts on the exciter diode wheel.

SAFETY SIGNIFICANCE:

There were no actual safety consequences associated with this event. Automatic protection and safeguards systems functioned as required following the trip. Plant parameters such as primary temperature, pressure and S/G level were restored to normal and the plant was stabilized in mode 3 (Hot Standby).

This condition is being reported per 10CFR50.73.a.2.iv as an unplanned Engineered Safety Feature/Reactor Protection System Actuation.

PREVIOUS SIMILAR EVENTS:

There have been no previous Turbine Trip/Reactor Trip events caused by an electrical failure in the Main Generator Exciter.

CORRECTIVE ACTIONS COMPLETED:

1. The exciter rotor assembly was replaced and inspections were performed to ensure no foreign material present in the exciter and exciter housing area.

CORRECTIVE ACTIONS PLANNED:

1. The plant's Foreign Material Exclusion (FME) Program will be strengthened with regards to the main generator exciter. This will be performed via a revision to Maintenance Procedure, PM-E0045 "Generator Exciter Cleaning, Inspection, and Testing" to specify cleanliness close-out requirements for the generator exciter. This will be completed by December 19, 1997.